Advances in Intelligent Systems and Computing 987

Wojciech Zamojski Jacek Mazurkiewicz Jarosław Sugier Tomasz Walkowiak Janusz Kacprzyk *Editors*

Engineering in Dependability of Computer Systems and Networks

Proceedings of the Fourteenth International Conference on Dependability of Computer Systems DepCoS-RELCOMEX, July 1—5, 2019, Brunów, Poland



Advances in Intelligent Systems and Computing

Volume 987

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ISSN 2194-5357 ISSN 2194-5365 (electronic) Advances in Intelligent Systems and Computing ISBN 978-3-030-19500-7 ISBN 978-3-030-19501-4 (eBook) https://doi.org/10.1007/978-3-030-19501-4

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Modelling and Safety Assessment of Programmable Platform Based Information and Control Systems Considering Hidden Physical and Design Faults

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Abstract. The information and control system (I&CS) of Nuclear Power Plant (NPP) is considered as a set of three independent hardware channels including on-line testing system. NPP I&C system's design on programmable platforms is rigidly tied to the V-model of the life cycle. Functional safety and availability during its life cycle are assessed using Markov models. Markov models are used to assess availability function and proof test period. The basic single-fragment model MICS01 contains an absorbing state in case of hidden faults and allows to evaluate risks of "hidden" unavailability. The MICS02 model simulates "migration" of states with undetected failures into states with detected faults. The results of Markov modeling (models MICS01 and MICS02) are compared to evaluate proof test period taking into account requirements for SIL3 level and limiting values of hidden fault probabilities.

Keywords: Functional safety modeling \cdot Information and control system \cdot Availability functions \cdot Undetected failure

1 Introduction

Nuclear Power Plant (NPP) information and control systems (I&CSs) such as reactor trip systems are safety critical systems. Very stringent requirements have been developed to such systems characteristics and life cycle processes. During the development cycle, it is possible to change the architecture of the NPP I&CS project, correct the parameters of its elements, program code and other changes. This affects the final characteristics of the system (quality, reliability, availability, safety, etc.).

The software architecture is modified on functions changing or on the detected software faults elimination. This affects code characteristics and failures and recovery

[©] Springer Nature Switzerland AG 2020 W. Zamojski et al. (Eds.): DepCoS-RELCOMEX 2019, AISC 987, pp. 264–273, 2020. https://doi.org/10.1007/978-3-030-19501-4_26

flow parameters. In papers [1–3], the mathematical apparatus of Markov and semi-Markov processes was used to study systems with variable indicators. In [4], a systematic approach to the construction of multi-fragment models was considered, which allows to structure complex systems models development. NPP I&CS design on programmable platforms such as RadICS [5] is rigidly tied to the V-model of the life cycle. During the development, verification and validation phases of the project, the results are recorded at intermediate stages. During the execution of the stage, it is possible to change both the system architecture and its parameters. Therefore, the primary developed safety assessment models need to be adjusted for changes and linked to the stages of the V model.

I&CS design of one of the NPP emergency protection system subsystems, previously considered in [5, 6], is investigated in this paper. Block diagram of the I&CS is shown in Fig. 1. It's one-version I&CS which implements logic of voting "2 out of 3". It includes three independent channels basing on programmable (FPGA) modules of the platform, in particular digital and analog input and output modules, logic module. Each channel is checked by the on-line testing systems for the dangerous failures presence. The hardware channels use the similar software (VHDL). If VHDL design faults occur it is equivalent to a common cause failure (CCF). To minimize CCF risk diversity approach (hardware-software components and process version redundancy) is applied [7, 8].

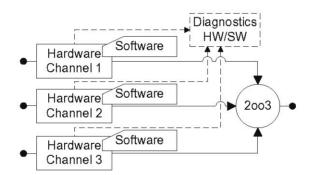


Fig. 1. I&CS reliability structural diagram [2].

Control system is characterized by the parameter of on-line test diagnostics coverage (DC). In the designed system, there are distinctions between the diagnostics of the hardware and software with the parameters DC_{HW} and DC_{SW} . Control is carried out continuously and the detected failures are eliminated immediately after detection.

The class of I&CS on programmable platforms RadICS [5, 8], which are used in NPP reactors emergency protection systems, is studied in this paper. Emergency protection systems belong to the group of complex objects, which are provided with regular proof tests. These tests are performed according to the regulations and should be treated as a separate process.

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